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Radiation Effects in ZrN

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Introduction

This work explores the effects of displacive radiation damage in ZrN, using ion beam irradiations.

- Next generation nuclear power reactors will use novel fuel types; promising improved **efficiency** and **safety**.
- **UN** is an advanced fuel concept possessing a high fissile U density, thermal conductivity and melting temperature.
- ZrN is the surrogate system of UN possessing similar chemical properties and identical structure.
- As fuel ages, radiation-induced damage accumulates, leading to undesirable material property changes.
- A better understanding of UN material behaviour and related mechanistic **processes** is needed if it is to be employed in next generation reactors.

Experimental objectives:

Investigate the mechanistic role of damage induced by the primary knock-on atom (PKA) of neutrons. Explore the effect of coarse-grained (cg) and nanocrystalline (nc) grain size on radiation tolerance





- Irradiation
- Ar⁺ keV irradiation (UKNIBC University of Surrey)
- Midrange doses (~250 nm): 1, 5, 10, 50,

Correlate microstructural modifications to changes in bulk material properties.





Nanoindentation



XRD

Methods

- 100 dpa
- Beam current: $0.05 0.17 \mu A$
- Max fluence: 1 x 10¹⁷ ions/cm²

Results

TEM



corresponding colours have been applied to help guide the eye

References

[1] Matzke, H. & Kinoshita, M. (1997) Journal of Nuclear Materials, 247, pp. 108-115. [2] Terricabras, A. J. et al. (2022) Journal of Nuclear Materials, 563, 153643. [3] Toth, L. E. (1971) Transition Metal Carbides and Nitrides, Academic Press, New York.

TEM characterisation of 600 keV Ar irradiated nc-ZrN

Influences of electronic to nuclear stopping ratios on radiation effects in cg-ZrN, nc-ZrN and UN. Currently analysing: 92 MeV Xe, 30 MeV Au, 9 MeV Si, 7 MeV Au, 1.5 MeV Si.